

Prepared by Dr. PK



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DEPARTMENT OF SCIENCES, M.Sc. (PHYSICS)
IV SEMESTER - END SEMESTER EXAMINATIONS, JUNE 2017

Make up

SUBJECT: NUCLEAR PHYSICS III [PHY-708.6]

(REVISED CREDIT SYSTEM)

Time: 3 Hours

Date:

MAX. MARKS: 50

Note: (i) Answer Any FIVE full questions. Each sub questions carries FIVE marks.

- Explain mirror nuclei method of measurement of nuclear size.
 - Explain the importance of electric quadrupole moment of a nucleus with example.
- Explain spatial distribution of slowing down density according to continuous slowing down model. What is meant by "age" of a neutron in a neutron multiplying system?
 - Show that critical energy of deformation for causing fission is linear function of the parameter Z^2/A for light nuclei.
- What is the Lawson's criterion for nuclear fusion reaction?
 - Obtain minimum critical radius for a reactor with square geometry in terms of critical buckling using one group equation.
- What is "Reflector Savings" with reference to a neutron multiplying system? Explain with an example.
 - Determine the infinite multiplication factor of a uniform mixture of uranium-235 and beryllium oxide in the atomic ratio of 1 to 10000. The value of σ_a for beryllium oxide is 0.010 barn. The resonance escape probability and the fast fission factor may be taken to be unity. η for uranium-235 is 2.06.
- Derive four factor formula for a thermal reactor.

- (b) Assuming that the energy released per fission of $^{235}\text{U}_{92}$ is 200MeV, calculate the number of fission processes that should occur per second in a nuclear reactor to operate at a power level of 20,000kW. What is the corresponding rate of consumption of $^{235}\text{U}_{92}$.
6. (a) How to produce transuranic element Plutonium ($Z=94$) isotope and mention its properties. Write down the outer electronic configuration for the same element.
- (b) A reactor core contains fuel and moderator [$\Sigma_s = 0.64 \text{ cm}^{-1}$, $\zeta = 0.17$]. The thermal neutron flux is $2 \times 10^{12} \text{ neutrons}/(\text{cm}^2)(\text{sec})$ and Σ_a for thermal neutrons in the fuel is 0.005 cm^{-1} ; for each thermal neutrons absorbed, 1.7 fission neutrons are produced. Estimate the epithermal neutron flux per unit lethargy interval.
